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# Active Neutron Measurement Techniques

## SEE LANL

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# Processes that generate neutrons

## ■ Fission

### — *Spontaneous*

- Nucleus splits all by itself, randomly
- $^{238}\text{U}$ ,  $^{238}\text{Pu}$ ,  $^{240}\text{Pu}$ ,  $^{242}\text{Pu}$ , Cm, Cf

### — *Induced*

- Heavy isotope absorbs neutron, causing it to split → Multiplication
- $^{235}\text{U}$ ,  $^{239}\text{Pu}$ ,  $^{233}\text{U}$

### — Bursts of 0-8 time-correlated neutrons emitted

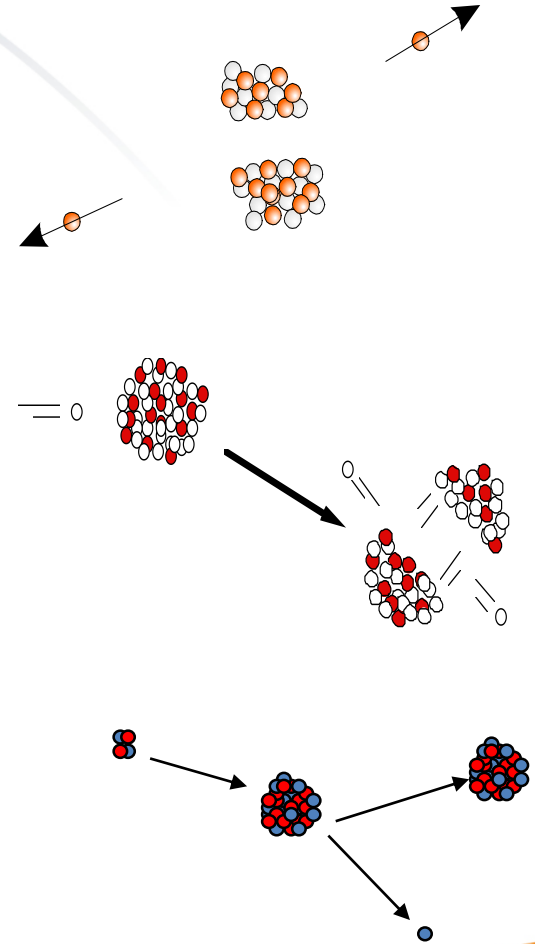
## ■ $(\alpha, n)$ reactions

- Helium nucleus reacts with light element nucleus, generating a single neutron and a new light element
- Neutron emission depends on item composition

## ■ Less common reactions

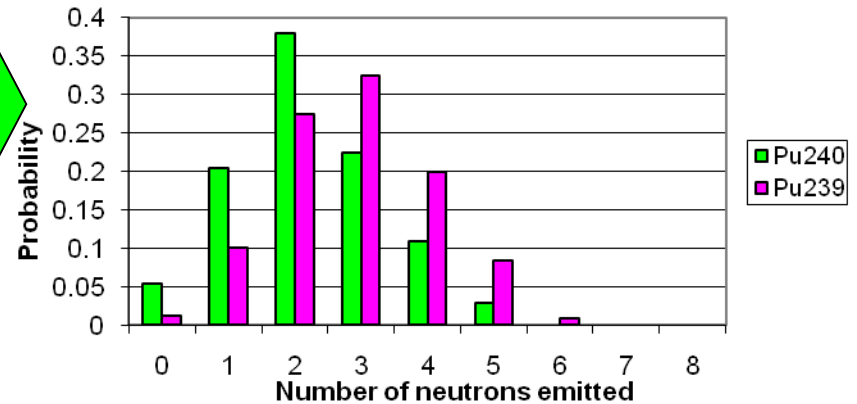
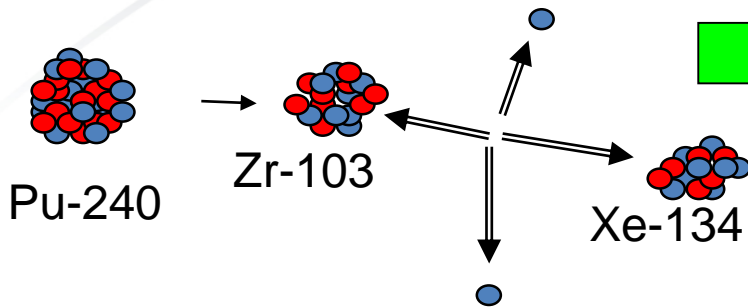
- Cosmic rays,  $(n, 2n)$ ,  $(p, n)$ ,  $(\gamma, n)$

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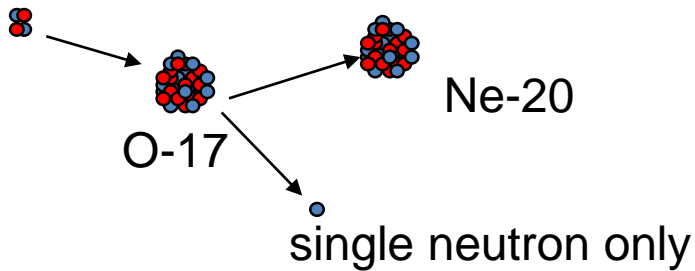


# Neutron Signatures

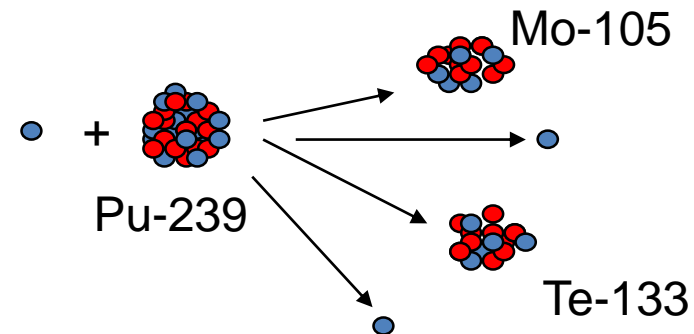
## Spontaneous fission



## (alpha,n) neutrons



## Induced fission

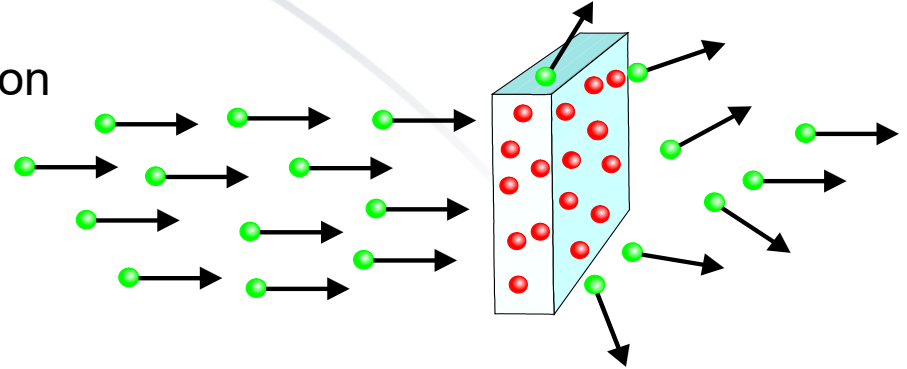


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# Basic neutron interactions

## ■ Scattering

- Neutron collides with a nucleus, causing neutron's speed and direction to change, but leaving the properties of the nucleus the same as before the interaction



## ■ Moderation

- Neutron scattering process by which a neutron collides with matter and loses energy
  - i.e. 2 MeV to 0.025 eV
- Best moderation when neutron collides with nuclei of similar mass
  - e.g. Water, polyethylene, other hydrogenous materials,  $m_H \sim m_n$

## ■ Absorption

- Neutron is absorbed, yielding an excited nucleus, which de-excites through the release of something else, like a proton or a gamma ray
  - e.g.  $(n,\gamma)$ ,  $(n,p)$ , and fission reactions.

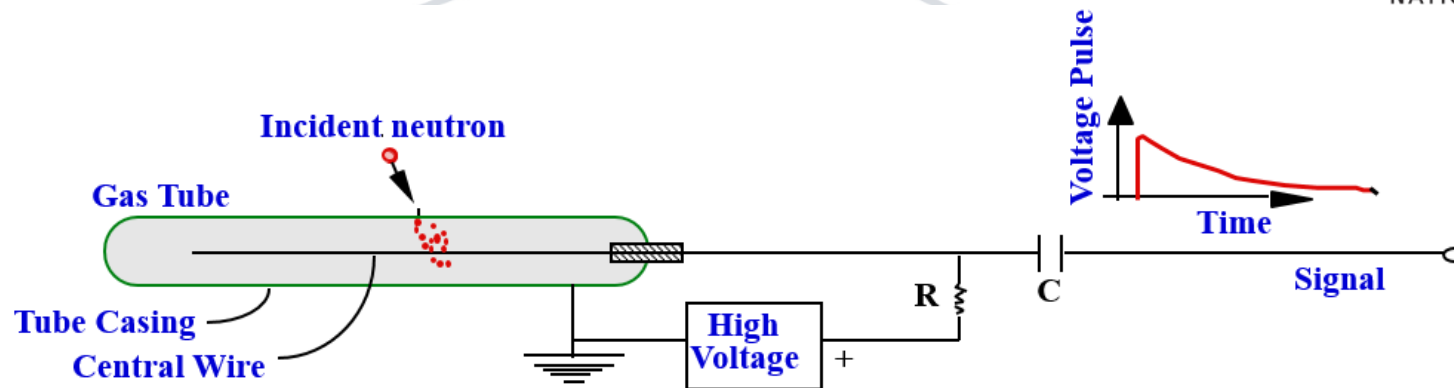
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# Why measure neutrons?

- Neutron rates are related to the amount of fissionable material. (Pu, U, etc. – what we need to safeguard)
- Highly penetrating.
  - Low rate of interaction with matter.
  - Can measure entire volume of item.
  - Can measure large-volume items. Gamma rays are limited (typically) to smaller items. (“Skin thickness”)
- Insensitive to interference by other gamma-emitting radionuclides (unless a  $(\gamma, n)$  source)

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# Neutron detector design



- Uses  $^3\text{He}$  tubes embedded in moderating material (polyethylene).
  - Fission neutrons emitted at MeV energies
  - $^3\text{He}$  tubes are most sensitive to low energy “thermal” neutrons  $E_{\text{ave}} \sim .025\text{eV}$
- Releases charge which is collected by gas tube.
- Detectors produce a distribution of electrical pulses.
- Electronics amplify the pulses, sets threshold, and converts pulses above threshold to digital pulses.

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# Instrument Choices

## ➤ Passive Assay

- Used for spontaneous fission
- Count neutrons produced by sample.
- Plutonium Assay:  $^{240}\text{Pu}_{\text{eff}}$
- $^{239}\text{Pu}$  inferred from isotopics.
- High Level Neutron Coincidence Counter (requires representative standards)
- Neutron Multiplicity Counter (applicable to wide range of material)

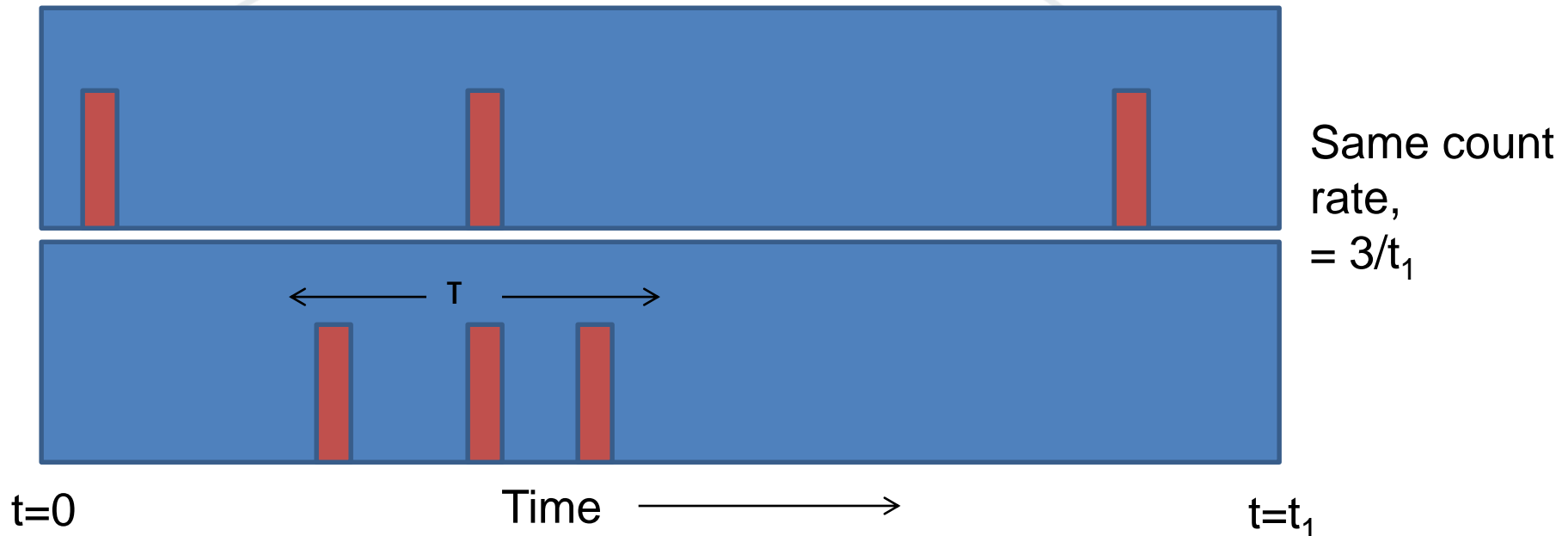
## ➤ Active Assay

- Used for induced fission when there is no (small) spontaneous fission
- $^{239}\text{Pu}$ ,  $^{235}\text{U}$
- Count neutrons induced by source.
- Active Well Coincidence Counter
- Uranium Neutron Coincidence Collar

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# Correlated Pulses

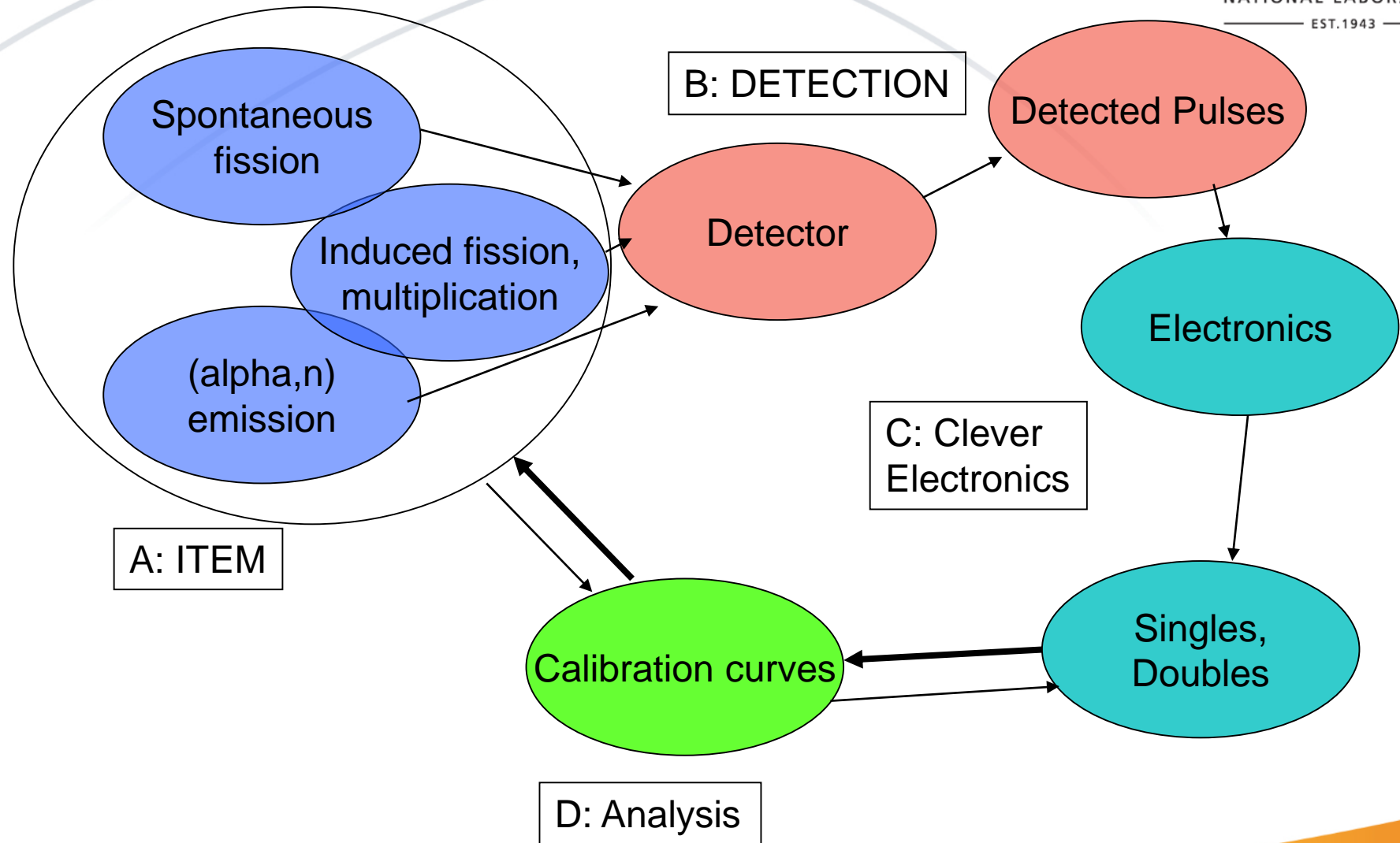
Two Pulse streams:



- Pulses from one fission event are clustered together in time.
- Pulses from unrelated events are distributed randomly in time
- We want a device to distinguish between these two different pulse streams

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# Coincidence counting process



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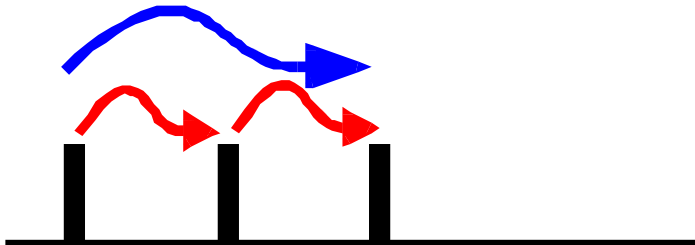
# Neutron coincidence counting



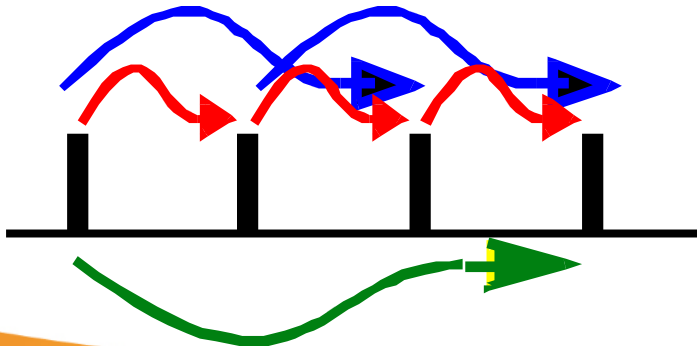
1 Pulse, 0 Coincidences



2 Pulses, 1 Coincidence



3 Pulses, 3 Coincidences

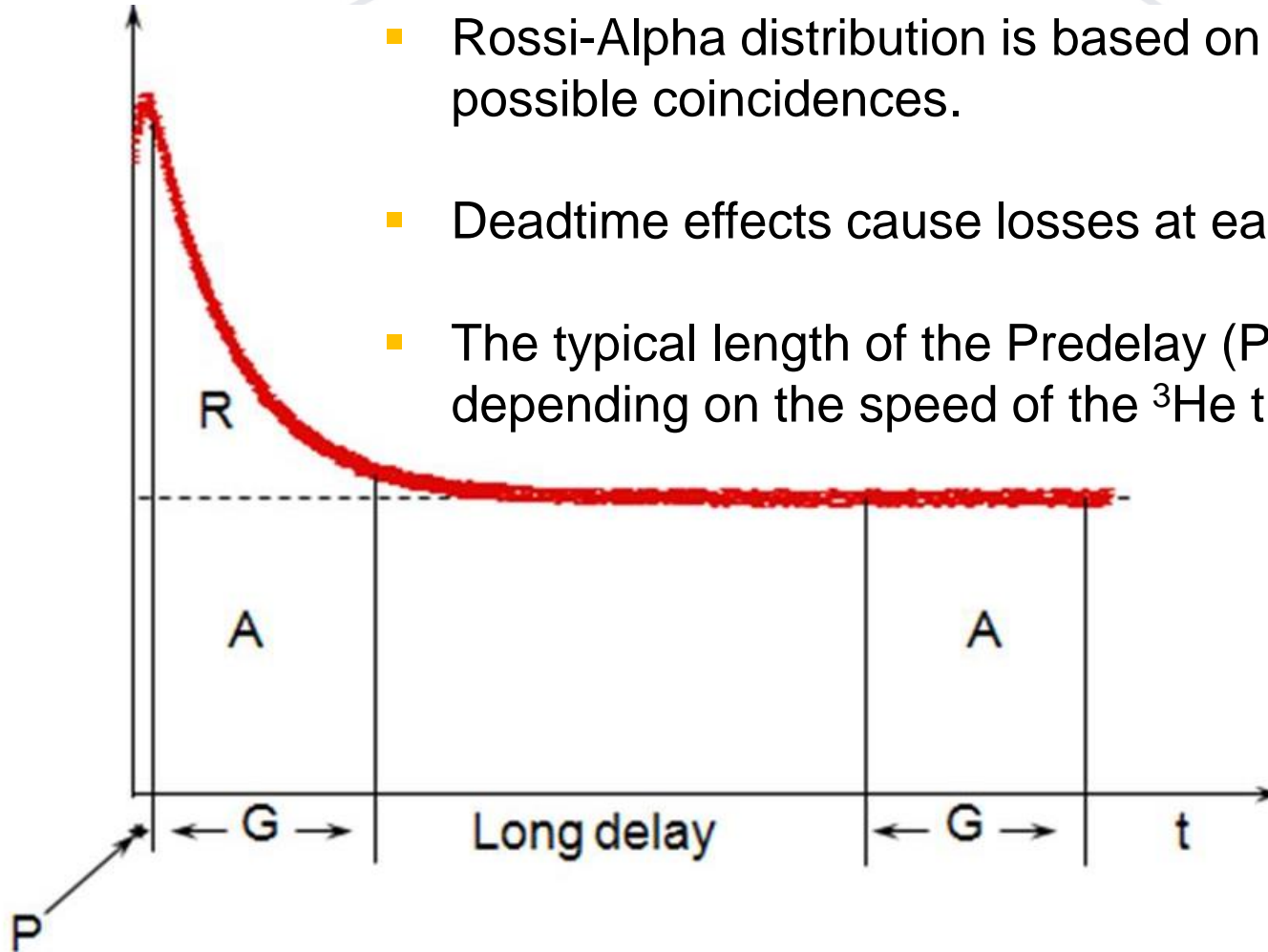


4 Pulses, 6 Coincidences

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# Rossi-Alpha Distribution

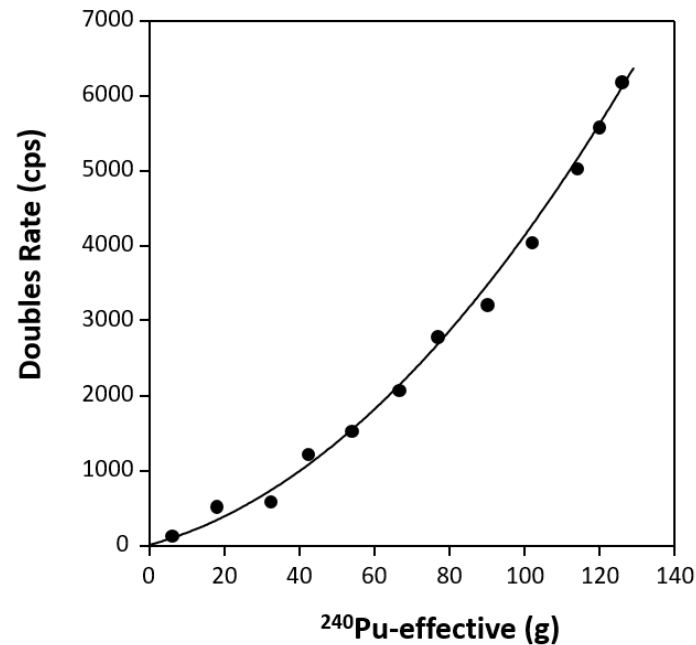
- Rossi-Alpha distribution is based on counting all possible coincidences.
- Deadtime effects cause losses at early times.
- The typical length of the Predelay (P) is 2 to 4.5 ms depending on the speed of the  $^3\text{He}$  tube and amplifier.



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# Calibration Curve Method

## *Calibration Curve Method*



- Need representative items for each item type and careful calibration measurements.

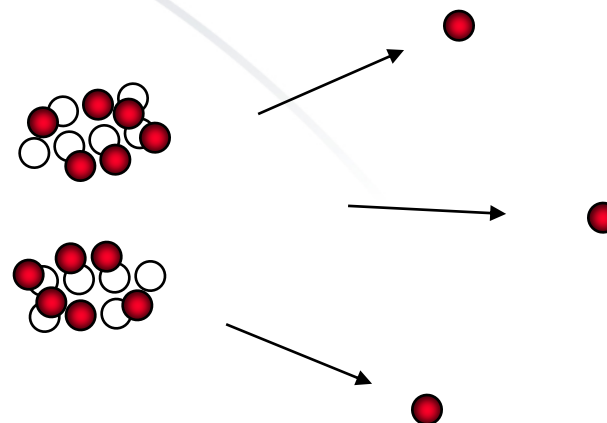
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# Active Neutron Coincidence Counting

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# Spontaneous Fission

<u>Nuclide</u>	<u>Specific Intensity [n/(g.s)]</u>
$^{234}\text{U}$	0.005
$^{235}\text{U}$	0.0003
$^{236}\text{U}$	0.0055
$^{238}\text{U}$	0.0136
$^{238}\text{Pu}$	2590.
$^{239}\text{Pu}$	0.022
$^{240}\text{Pu}$	1020.
$^{241}\text{Pu}$	~0.05
$^{242}\text{Pu}$	1720.
$^{241}\text{Am}$	1.18
$^{252}\text{Cf}$	2.34E+12

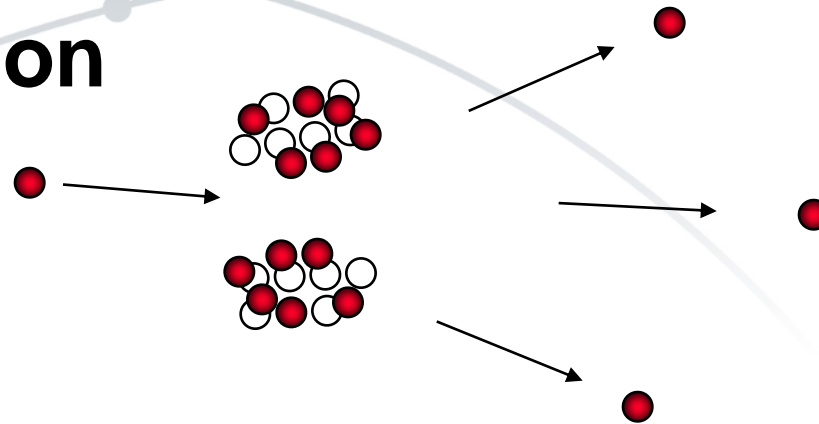


Uranium spontaneous fission emission rate is very small and is generally not useful for NDA except for large quantities of  $^{238}\text{U}$

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# Induced Fission

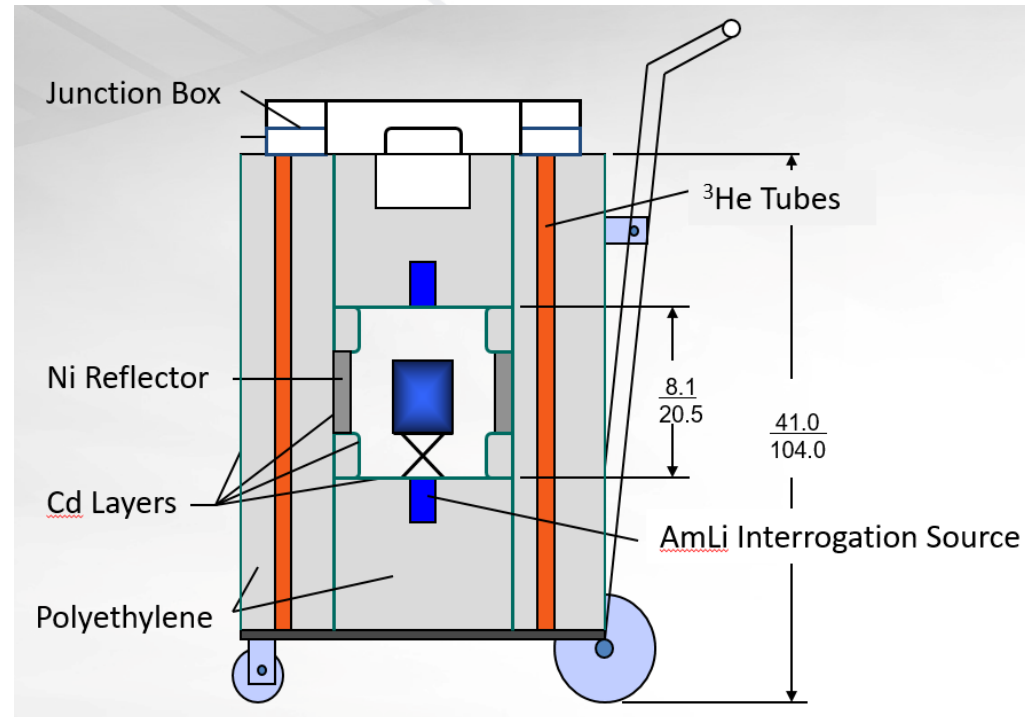


- Induced fission is the primary method for uranium assay.
- Uses AmLi interrogation sources because they produce only random neutrons and have a low energy spectrum that will only induce fission in  $^{235}\text{U}$
- Neutron emission occurs in bursts (0-8)
- The coincidence rate is related to the  $^{235}\text{U}$  mass.
- Coincidence rate is dependent on item properties: geometry, item composition, density.

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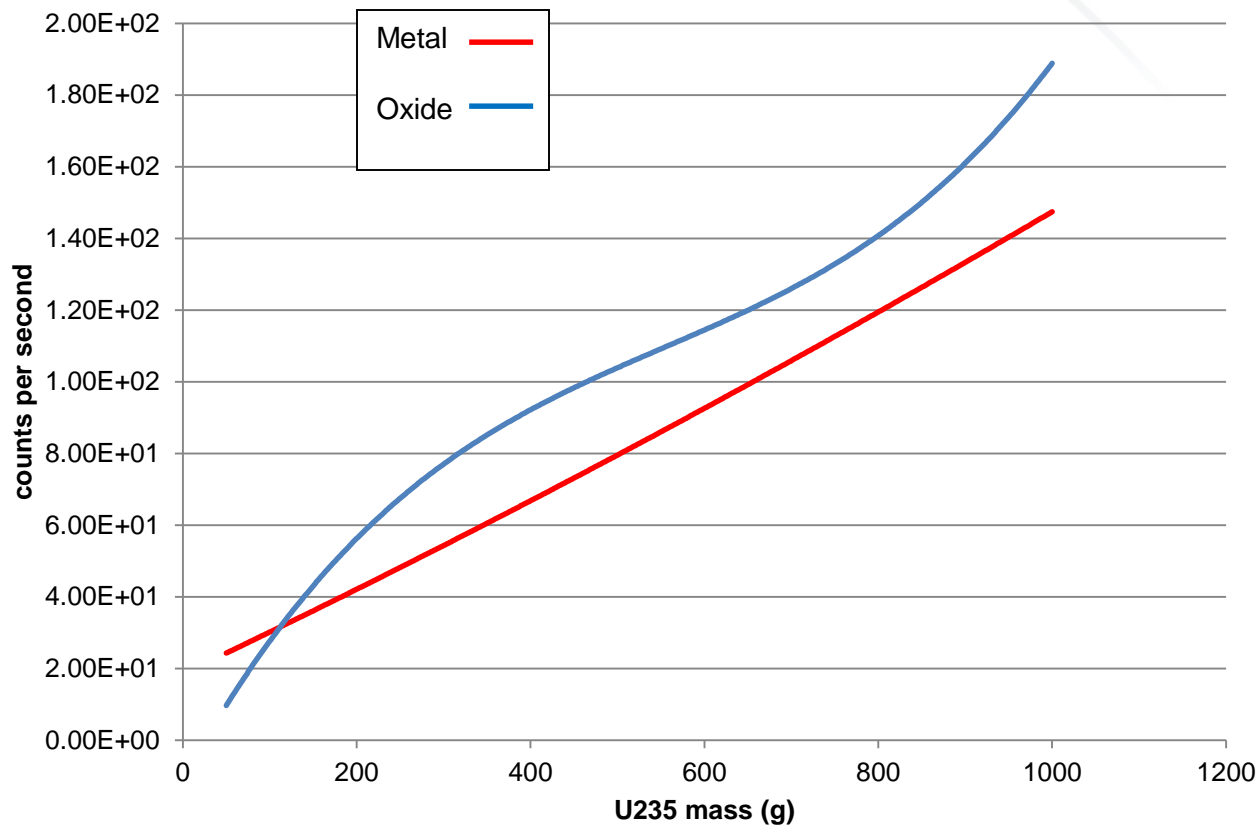
# Active Well Coincidence Counter (AWCC)

- Designed in 1984 (Mod II)
- Assay range of few gram to several kg of  $^{235}\text{U}$
- Can be used in passive or active modes
- Portable
- Good efficiency – 42  $^3\text{He}$  tubes
- Uses two AmLi sources for uniform interrogation
- Several cavity configurations for optimization of performance



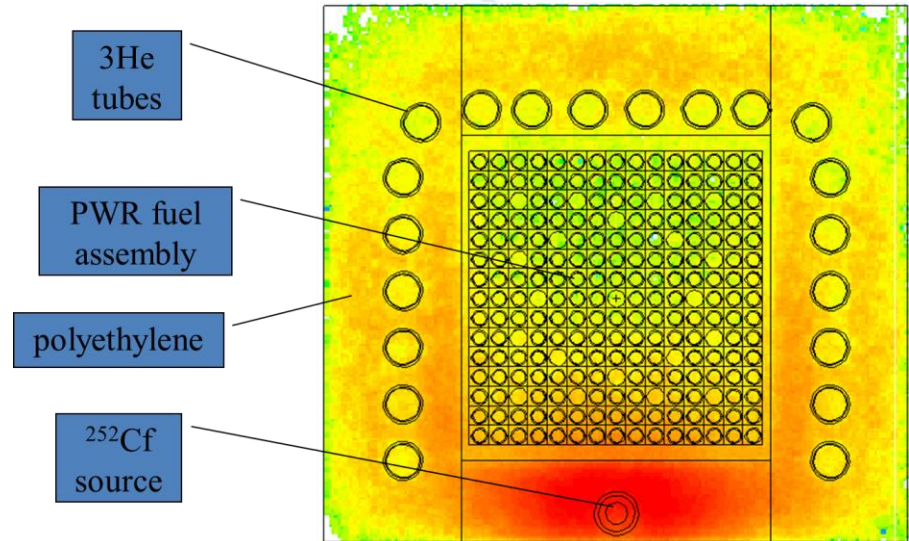
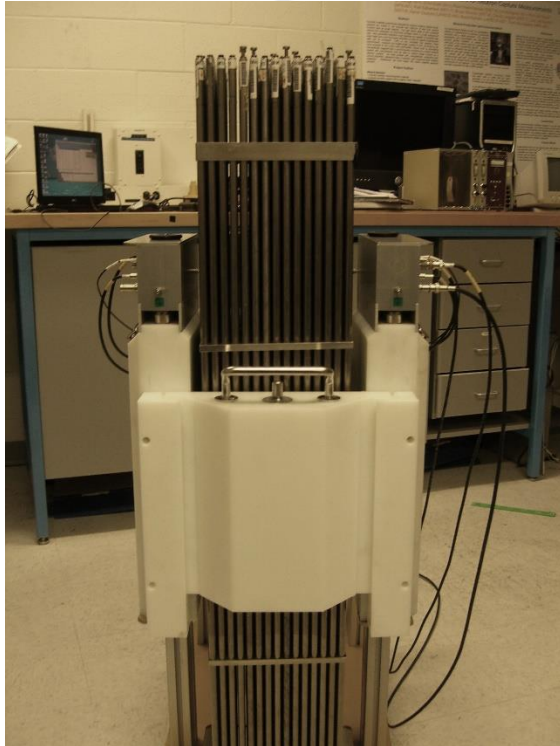
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# Sample calibration curve comparison



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# Uranium Neutron Coincidence Collar (UNCL)



- Used for verification of fresh fuel assemblies (BWR and PWR)
- Response cross-calibrated to an absolute calibration curve
  - Different calibration curves for BWR and PWR

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